



Overview of ASDEX Upgrade Results

Otto Gruber for the ASDEX Upgrade Team

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- 16 IPP, CAS, Hefei, China,
- 17 NCSR Demokritos, EURATOM Association-HELLAS, Athens, Greece

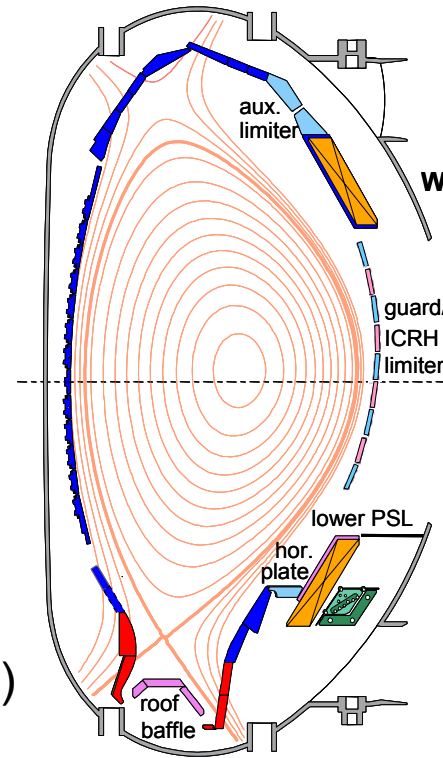


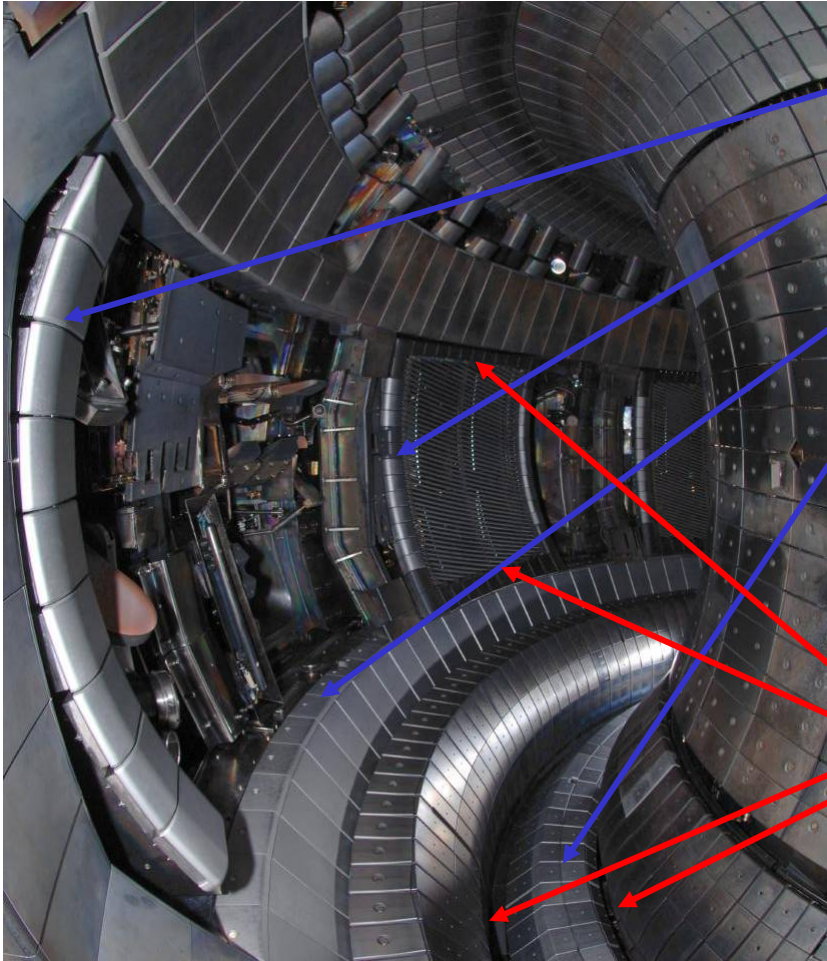
- **Main aim is to establish the physics base for ITER (and DEMO):**
 - consolidation of the 'standard' H-mode scenario
 - exploration of ,advanced' modes beyond the standard scenario
- **Understanding of physics elements**
 - transport
 - fast particles and NBCD
 - H-mode edge and ELM tailoring
 - disruption mitigation
 - MHD control with ECCD
 - tungsten wall and divertor operation
- **Integration into improved scenario beyond reference**
 - Improved H-mode (ITER Hybrid scenario)
 - ITER relevant digital CODAC system
- **Direct influence on ITER component design:**
 - PFC material, heating/CD systems, ECRF system
- **Strategy:** in close collaboration within EU fusion programme, ITPA TGs

AUG enhancements 2004-2006:



- towards a tungsten first wall: 36 m² or 85% of PFC area)
 - 2005: - all LFS limiters (water cooled)
 - roof baffle with thin W coating (<4 μm)
 - 2006: lower divertor target plates (200 μm W)
- 4 steerable ECRH mirrors installed
- first two-frequency gyrotron: leak after commissioning (≤1 MW / 10 s / 105 & 140 GHz)
- pellet injection systems
 - centrifuge (HFS launch capability, variable pellet size, < 1200 m/s)
 - blower gun (optimized for decoupling ELM pacing and refuelling)
- new CODAC commissioned
 - reduced cycle time <1.5ms
 - extended regime recognition & performance control
 - real-time diagnostics →replaces CAMACs





- in 2005 “thin” W coating of
 - 4 guard limiters at LFS (water cooled)
 - 8 ICRH antenna side limiters
 - top of bottom PSL
 - roof baffle

- in 2006
 - upper and lower ICRH limiters
 - W coated bottom target tiles (200 μm)

→full tungsten machine

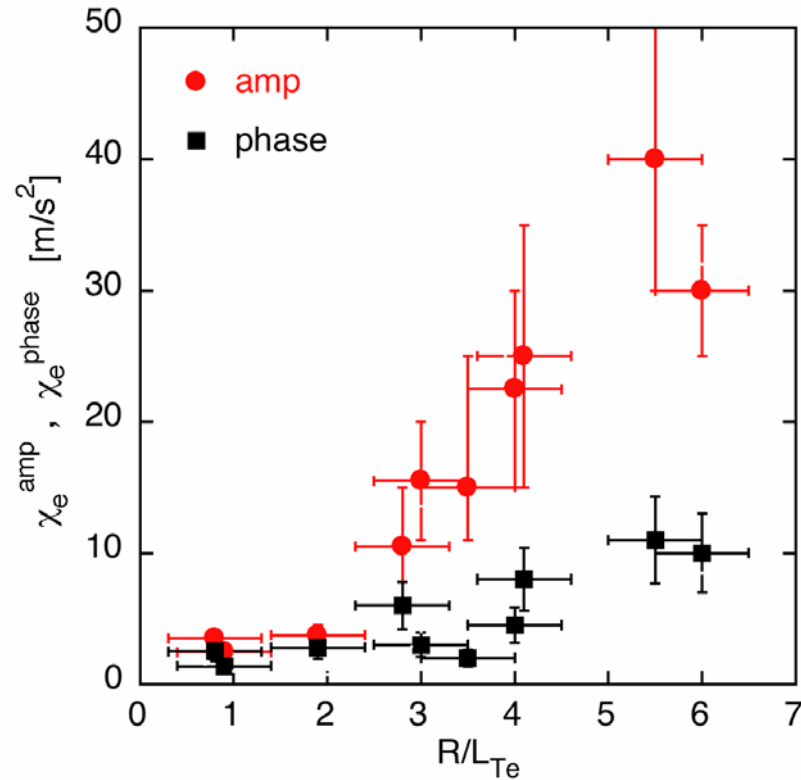
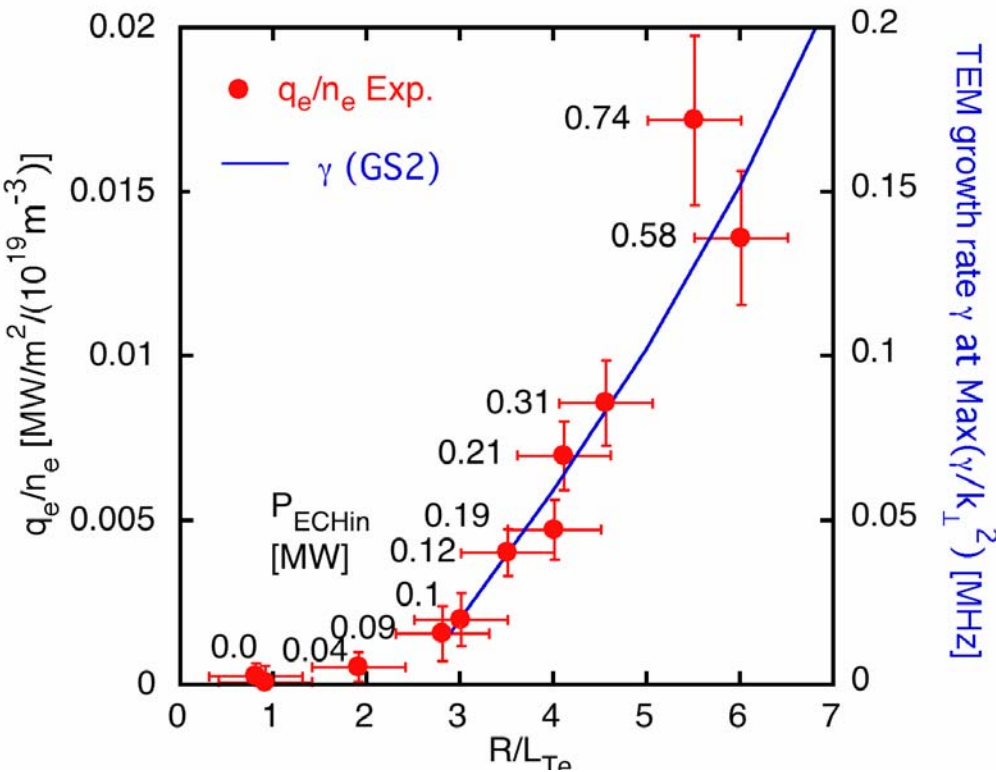
ITER start-up configuration ?

Understanding of anomalous transport → predictions

- response of different transport channels on heat and momentum input
- comparison with gyrokinetic simulations
- TEM and ITG turbulence dominate in different parameter regimes

Pure electron heating: **threshold for TEM at $R/L_{Te} \approx 3$**

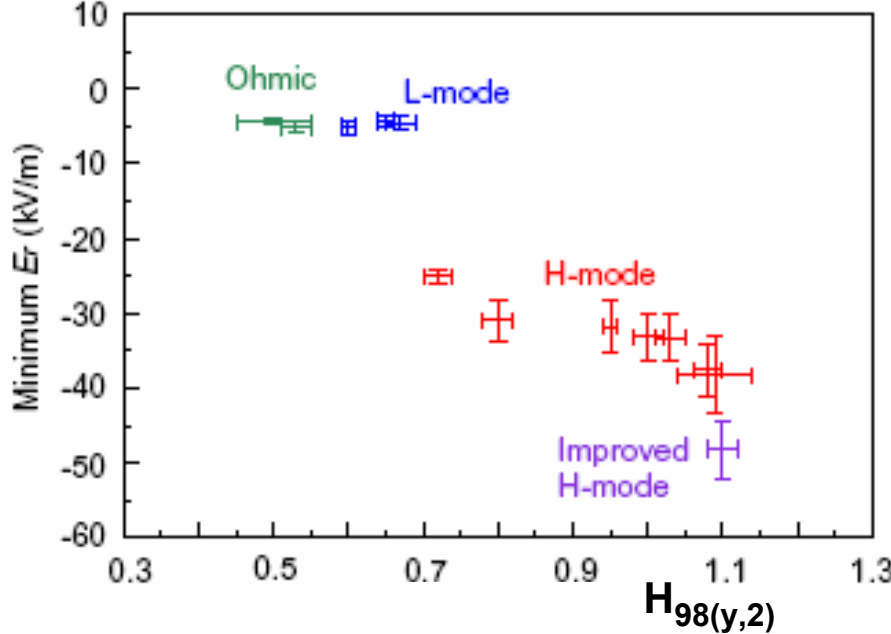
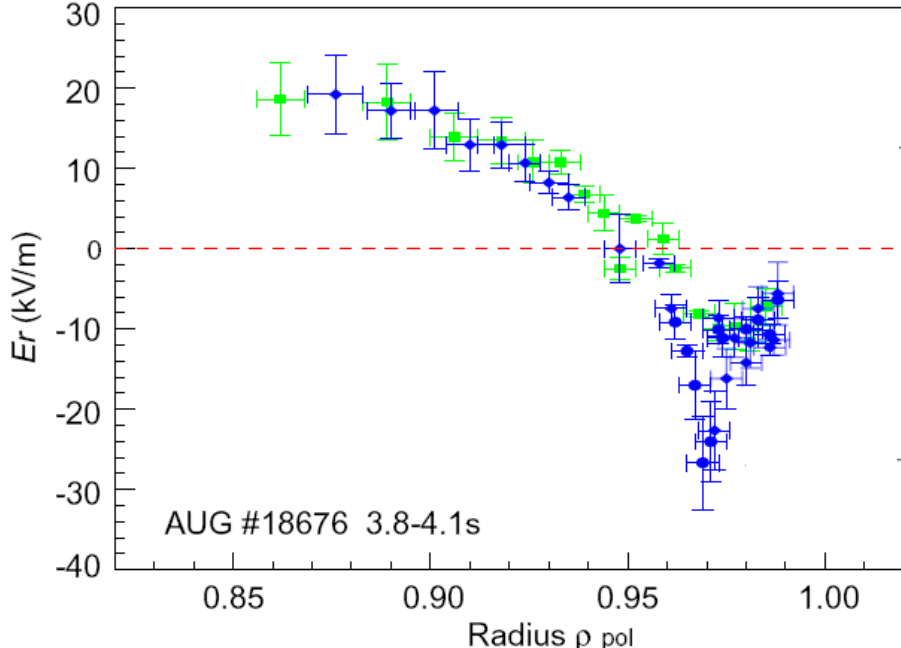
Angioni EX/8-5Rb



- power balance and heat pulse propagation show a transition through threshold

Transport: E_r transitions at plasma edge

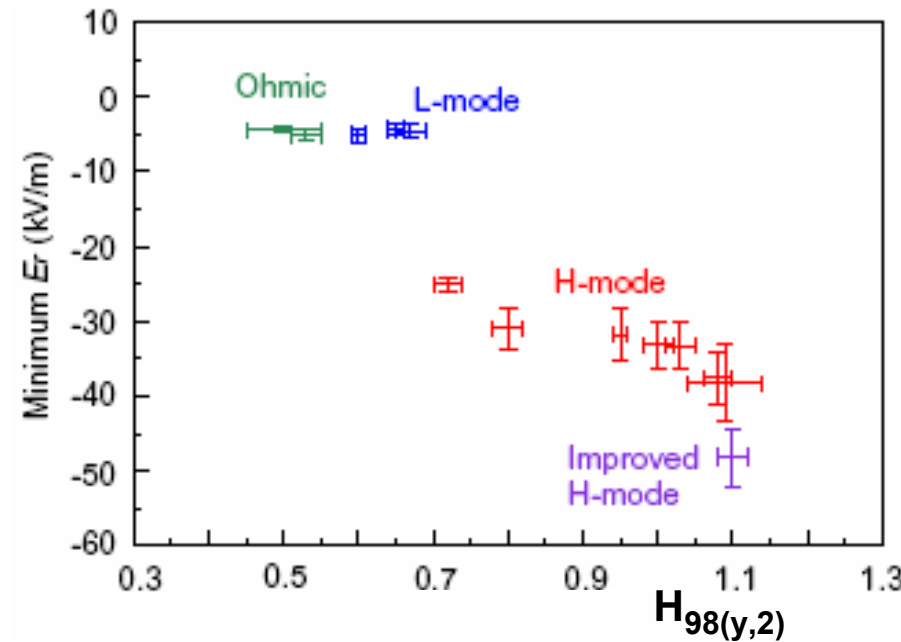
- **negative E_r well increases with confinement improvement**
 - coincides with H-mode barrier gradient
 - Doppler reflectometry



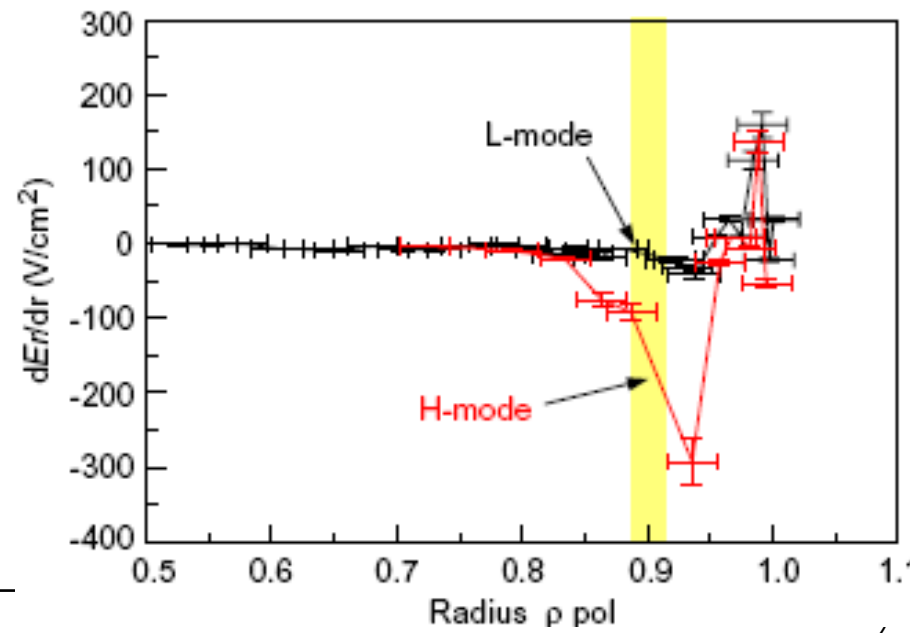
Conway EX/2-1

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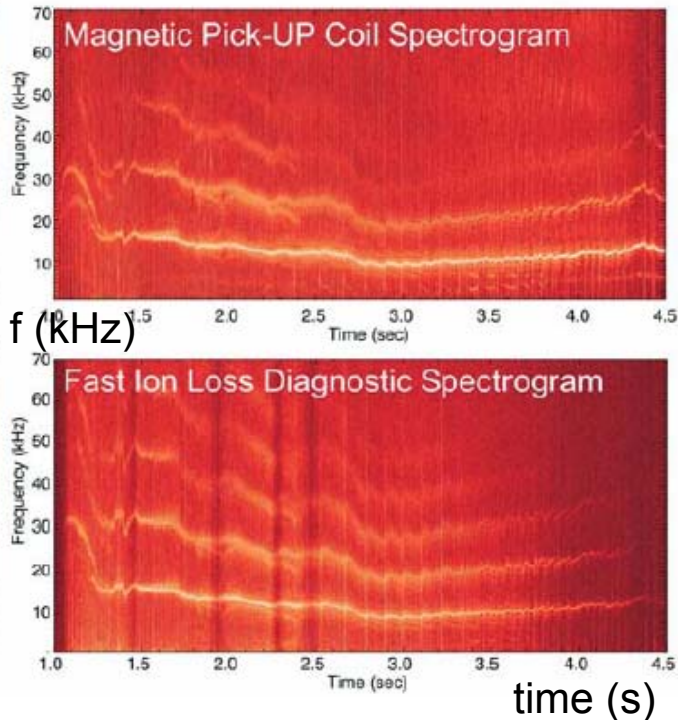


- **E_r shear enhanced as well**
 - 2 channel Doppler at fixed $\Delta f \sim 2\text{GHz}$
 - negative shear at pedestal increases with confinement
 - shear width $\leq 5\text{cm}$



Conway EX/2-1

- New: Fast Ion Loss Detector (FILD) with bandwidth of 1 MHz
- **frequency / phase correlations of fast ions with TAEs** (ICRH, ICRH beatwave)
- **fast particle losses correlated with low frequency MHD activity:**
NTMs, double tearing modes, ELMs

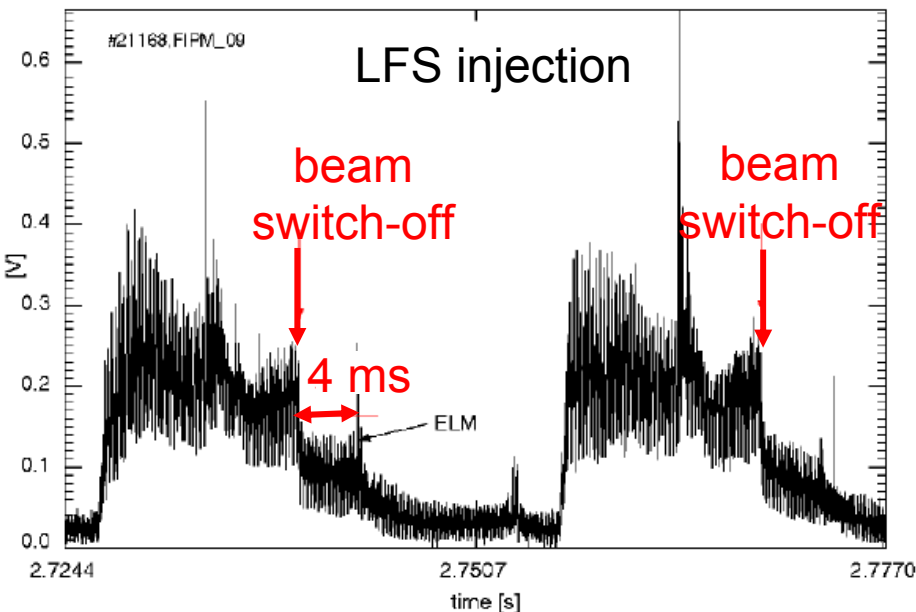


- slow MHD activity like NTM (+harmonics) induces fast particle losses
- FILD signal is modulated with rotating mode in fixed phase relation
- modulated NBI sources with different injection geometry \rightarrow origin of fast particles
- time scale of losses >100 toroidal orbit transits due to stochasticity of overlapping drift islands caused by the NTM
- NTM stabilization \rightarrow decrease of losses

Fast ion losses track the details of the mode

Fast particle interactions with large scale instabilities

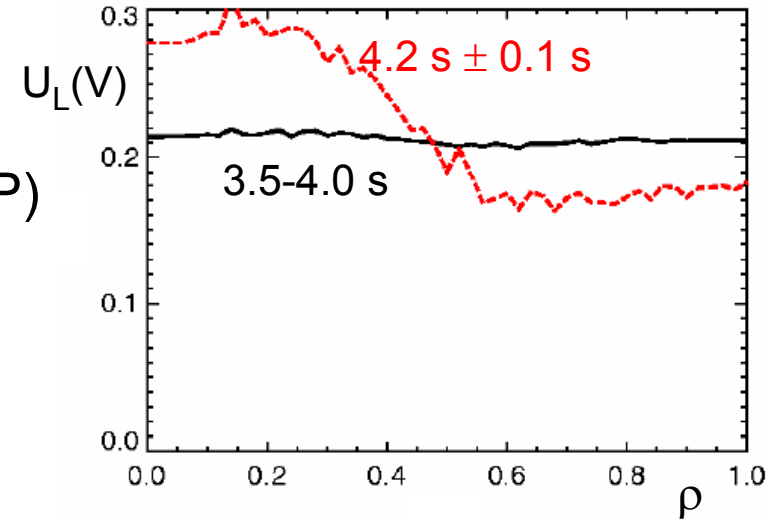
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- NTMs, double tearing modes, ELMs



- FILD signal caused by DTM
- response time depends on origin
- orbit drifts and slowing down determine delay

Unexpected broadening of NBI driven currents

- beyond a certain heating power, measured and predicted distributions of NBI driven currents deviate (MSE, TRANSP)
- electric field changes cannot be explained by current diffusion

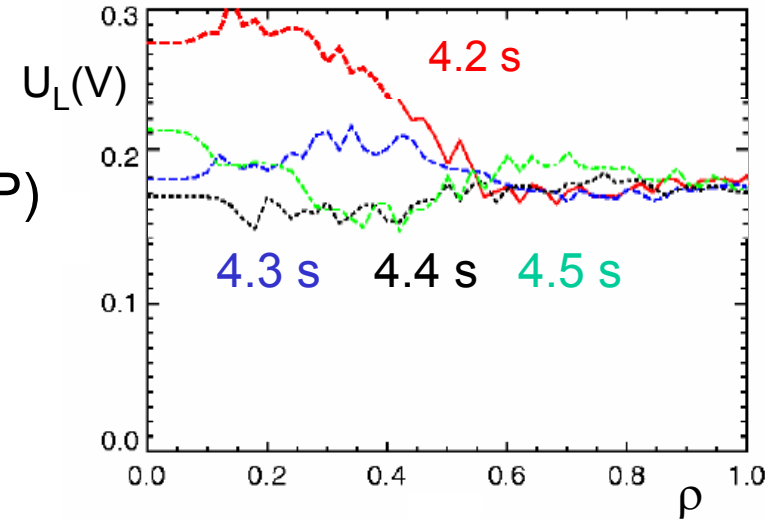


Switch on / off-axis at 4.1 s

Günter EX/6-1, McCarthy TH/P3-7

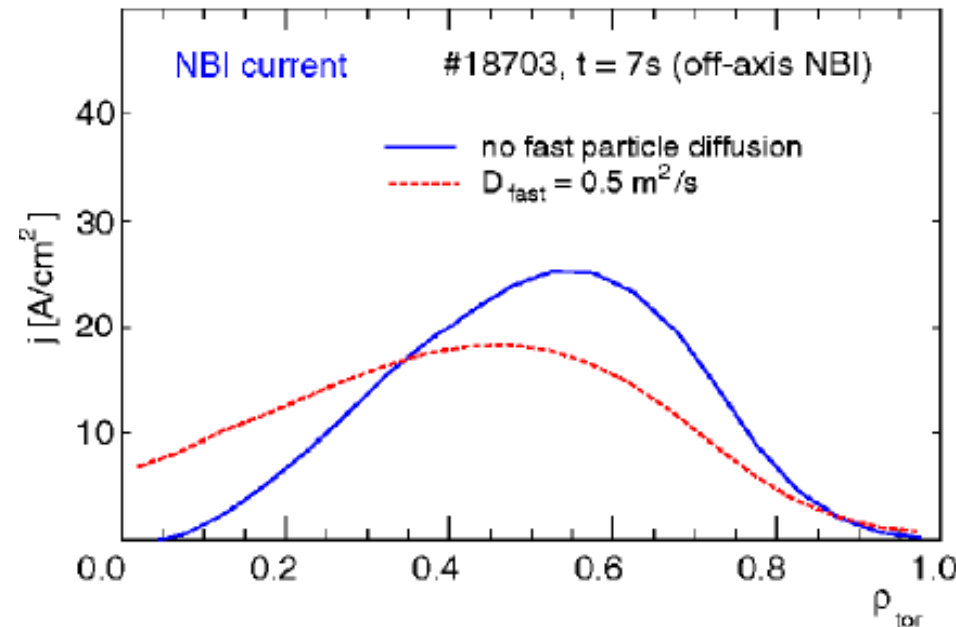
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- electric field changes cannot be explained by current diffusion



- **energetic particle diffusion driven by small-scale turbulence** (gyrokinetic code)
- redistribution of injected ions with $D_{fast} \approx 0.5 \text{ m}^2/\text{s}$

Switch on / off-axis at 4.1 s



Günter EX/6-1

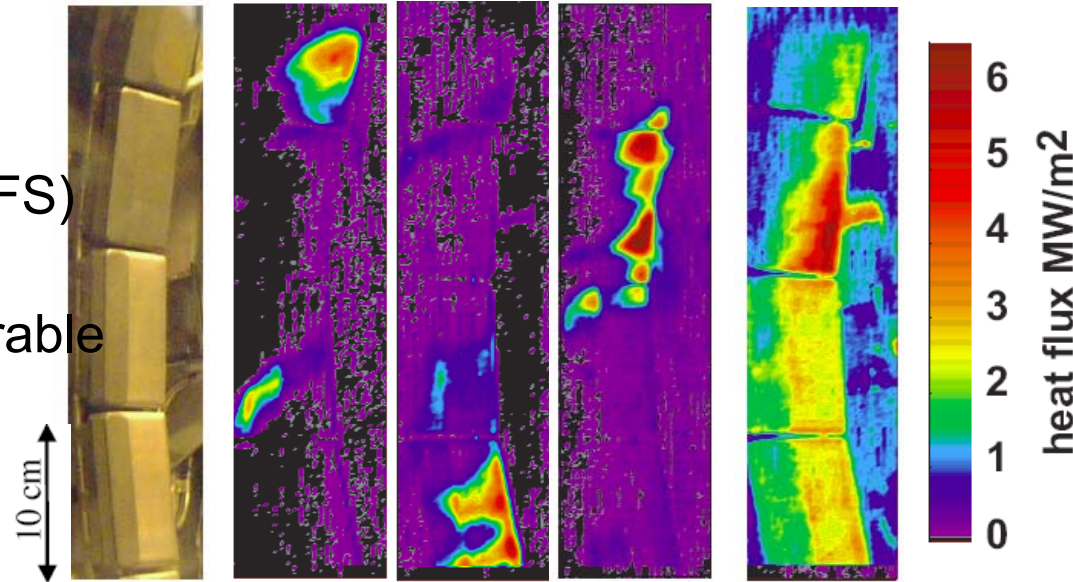
ELMs and disruptions

- most of the ELM and disruption energy is deposited in the divertor
- a smaller fraction goes to the main chamber wall

ITER: small ELM regimes / ELM pacemaking & disruption mitigation mandatory

ELMs:

- helical field aligned structures with a 3-6 cm spatial width and 3-6 km/s rotation velocity
- move radially far into the SOL (LFS)
- heat flux decay length 2-3 cm
- particle flux decay length comparable
- consistent with convective loss along field lines



Neuhauser EX/P8-2

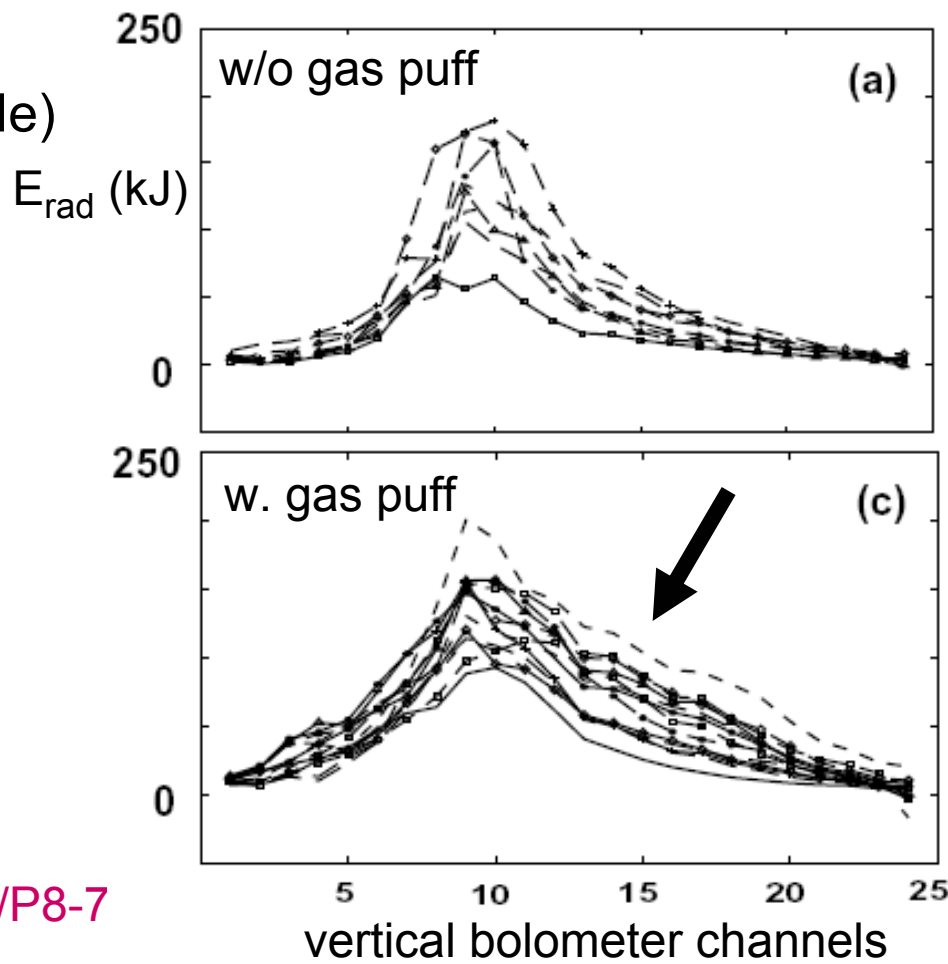
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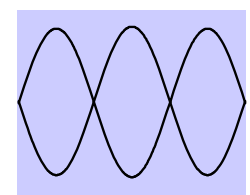
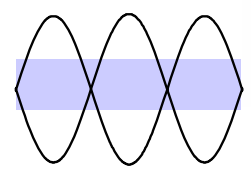
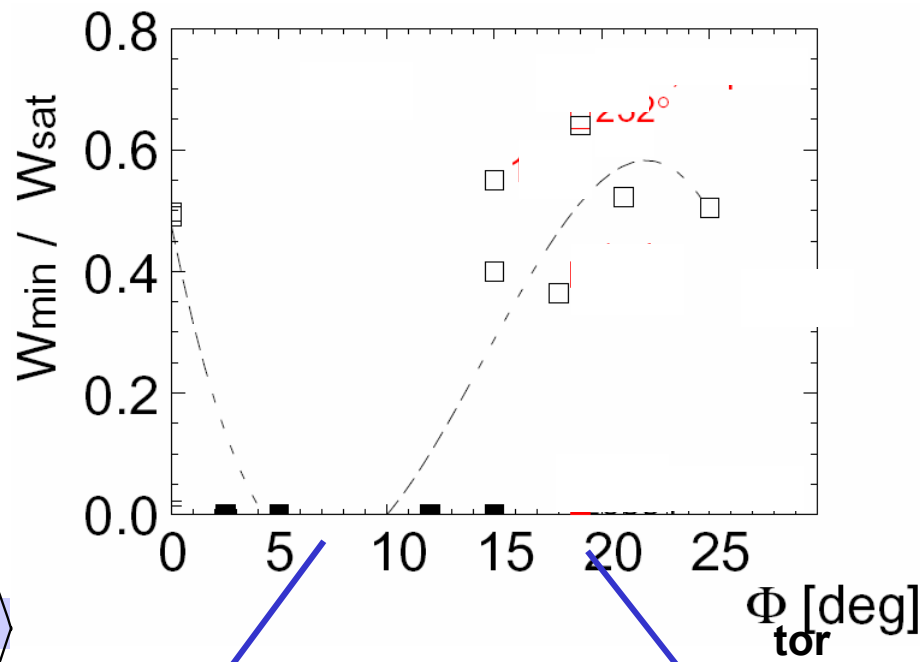
Disruptions:

- mitigation by puffing of noble gases (Ne) regularly used at AUG
- significant reduction of force loads on all structures
- divertor heat load mainly reduced by broader radiation and heat deposition profiles in divertor
- further optimization needed: higher gas pressure and amount



Pautasso EX/P8-7

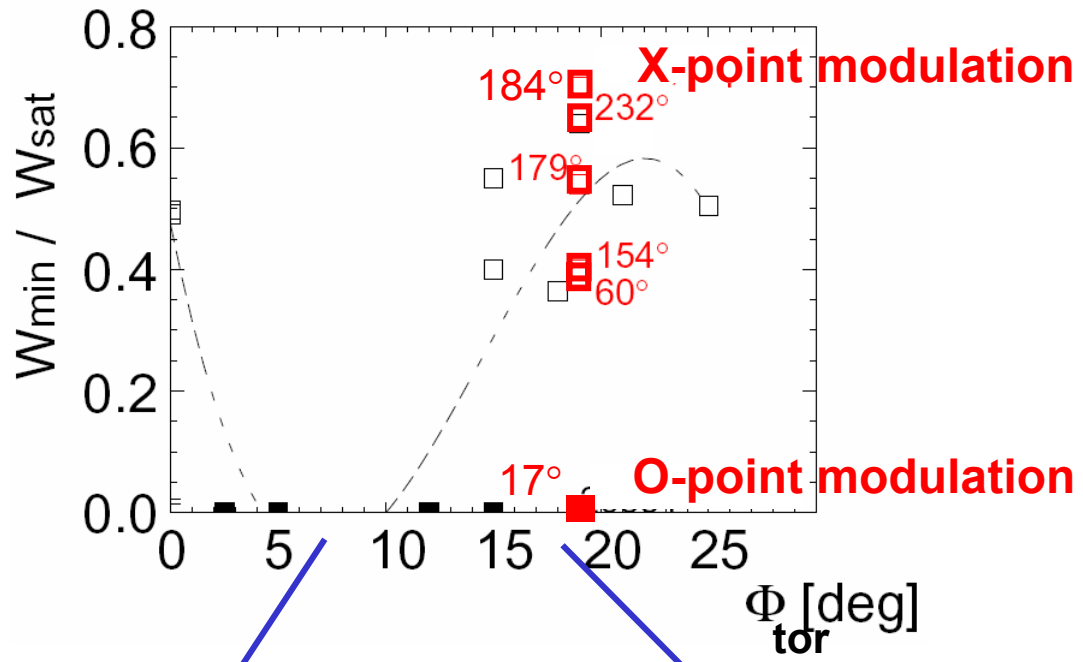
NTM stabilization: Influence of ECCD deposition width d



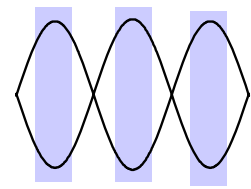
- **narrow deposition $W > 2d$:**
 - I_{ECCD} counts for helical CD
 - full stabilisation with dc ECCD of (3,2) and (2,1) NTMs
 - no advantage of phased ECCD

- **broad deposition $W < 2d$:**
 - mimics ITER $W_{marg} \sim \rho_{pol}$
 - I_{ECCD}/d^2 counts for dc ECCD
 - only partial dc stabilization
 - required current increases significantly
 - ⇒ **modulated ECCD required (at mode frequency / O-point injection)**

NTM stabilization: Modulated ECCD with broad deposition



Zohm EX/4-1Rb
Yu TH/P3-13

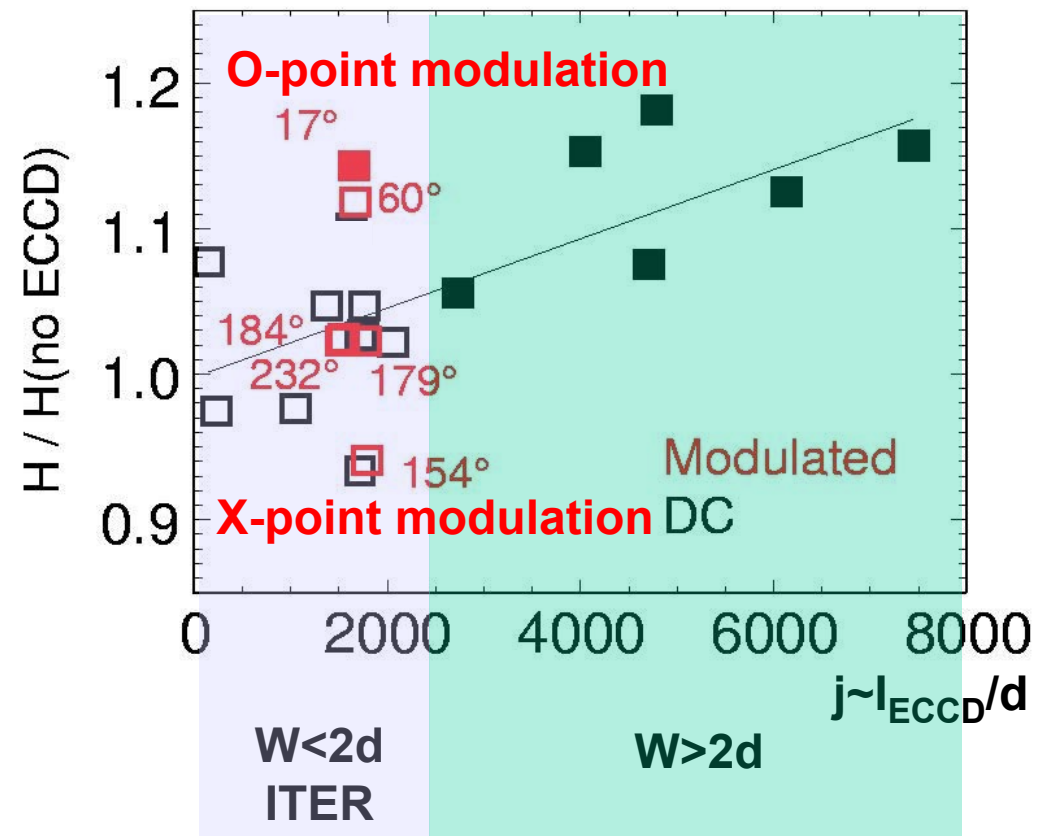


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 - mimics ITER $W_{marg} \sim \rho^*$
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⇒ **modulated ECCD required**
(at mode frequency / O-point injection)

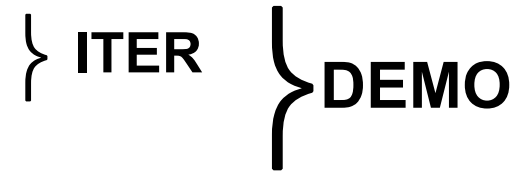
Confinement improvement



Zohm EX/4-1Rb

High-Z wall and divertor in ITER / DEMO

- pro:
- tritium co-deposition with carbon
 - erosion of low-Z material
 - neutron bombardment destructs graphite
- con:
- central radiation losses sets limit $c_W < \text{some } 10^{-5}$



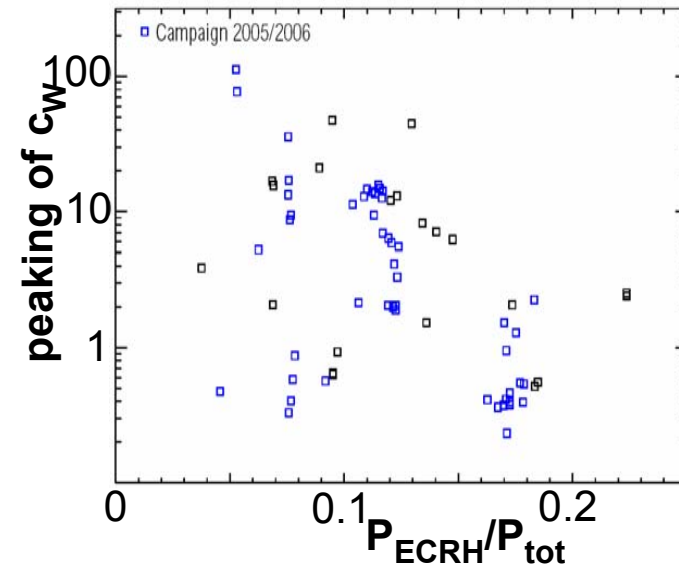
Sputtering source mainly from LFS limiters

- CX neutrals $\leftarrow T_e(\text{edge})$
- fast ions from NBI: depends on injection geometry
- antenna limiters with ICRH: 60-90% of W influx
 - sheath rectified E-fields accelerate impurities
- drastic enhancement of all sources during ELMs

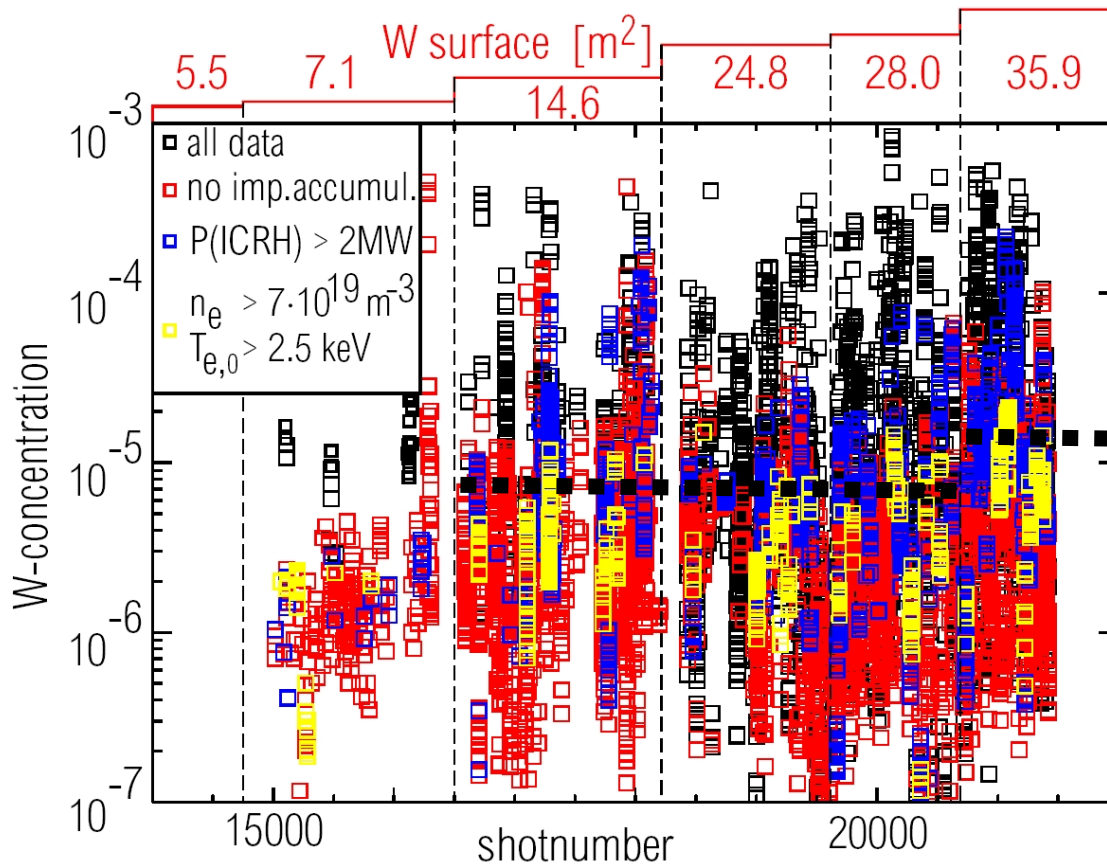
Dux EX/3-3Ra

Impurity transport

- H-mode barrier
 - ELM frequency control by pellet injection
- neoclassical inward pinch
- anomalous outward impurity transport enhanced by central heating (ICRH, ECRH)



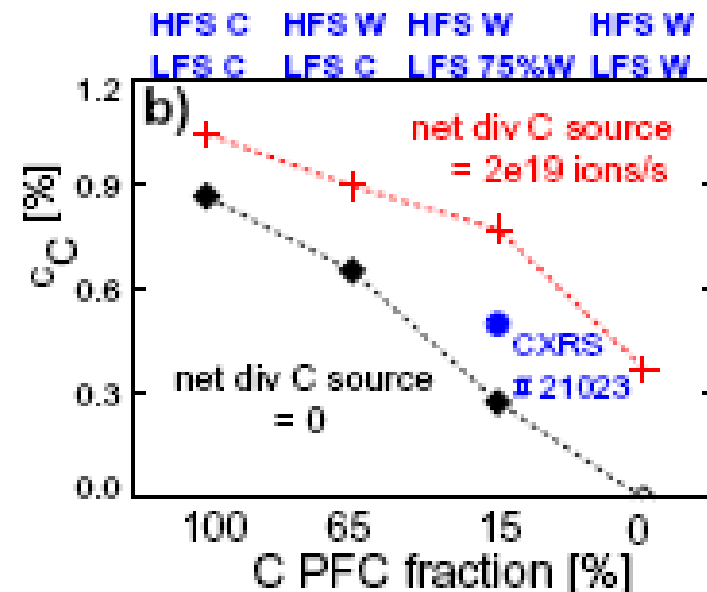
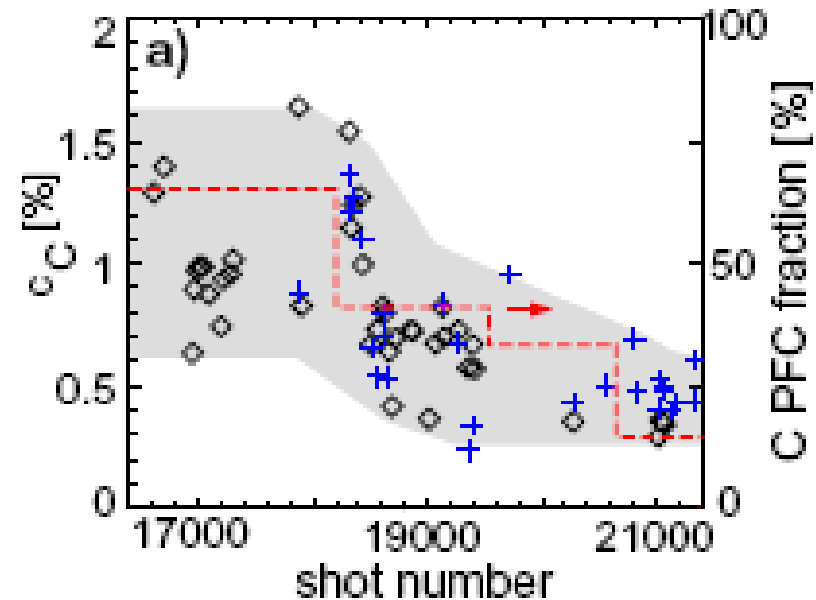
W wall: Long term evolution of W concentration



- wide distribution depending on plasma conditions: increase with W coverage, saturation of mean value around 10^{-5}
- reduced c_W at relevant central heating power and higher densities (ITER!)

W wall: Indications for transitions to W device

- reduction of C plasma content (standard H-mode discharge)
- Migration / transport model
 - slow evolution due to strong C recycling
 - C ,leaking' out of divertor important
 $1 \cdot 10^{19}$ atoms/s
 → remaining strong net erosion zone



Dux EX/3-3Ra

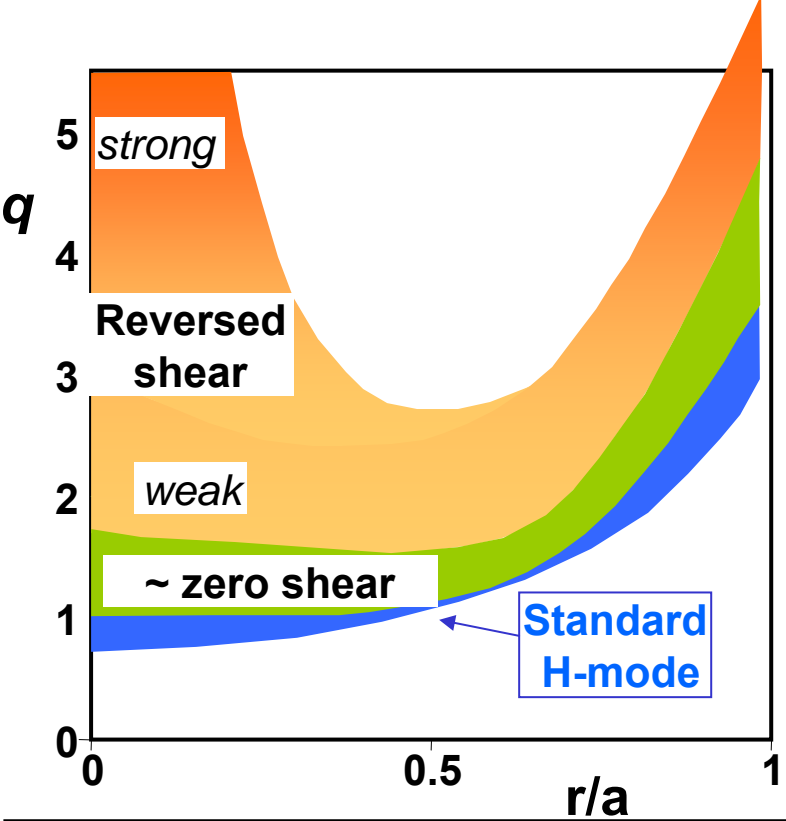
Noble gas retention / release:

Schmid EX/3-3Rb

Improved H-mode: Characterization of „advanced scenarios

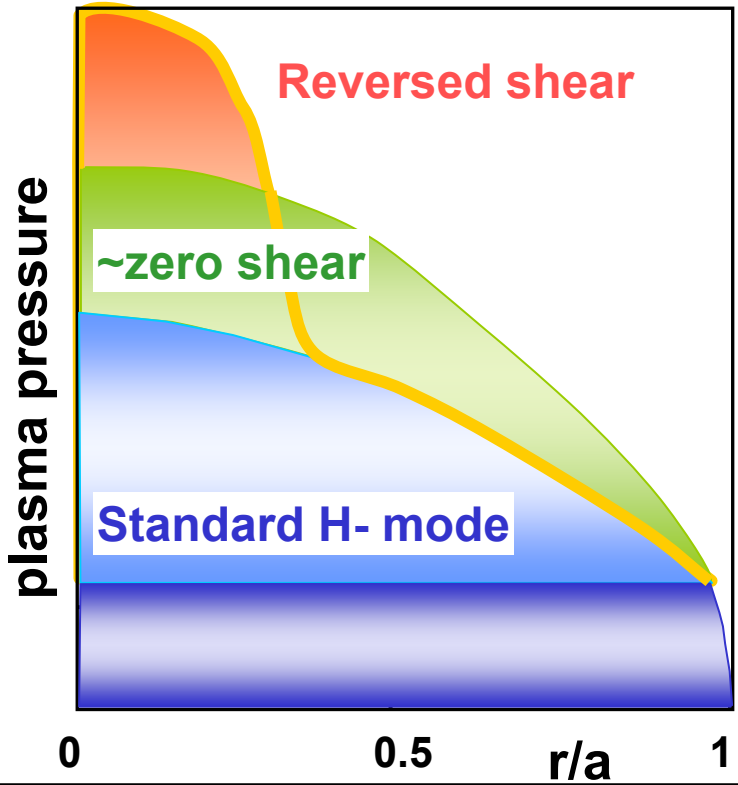
Zero shear, ‘hybrid’ discharges:

- elevated $q(0)$ above 1 desirable
- stationary with $\beta_N H/q_{95}^2$ up to 0.4
- high β -limit close to no wall limit
- substantial bootstrap fraction $\leq 50\%$
- no bifurcation, smooth evolution

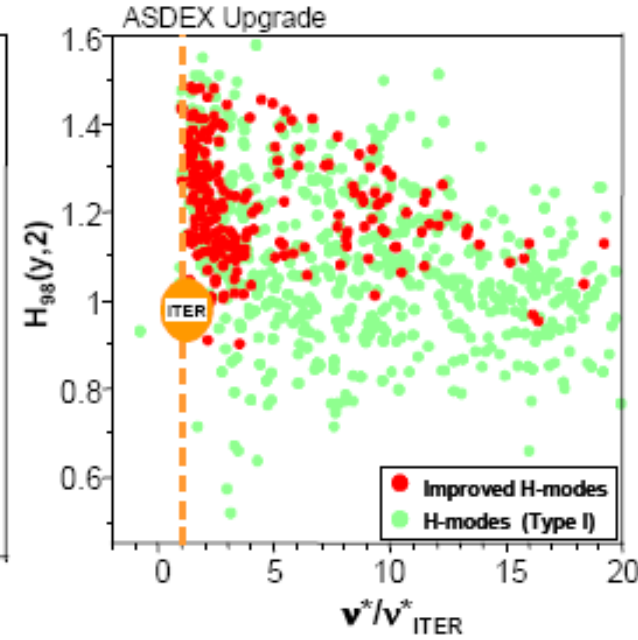
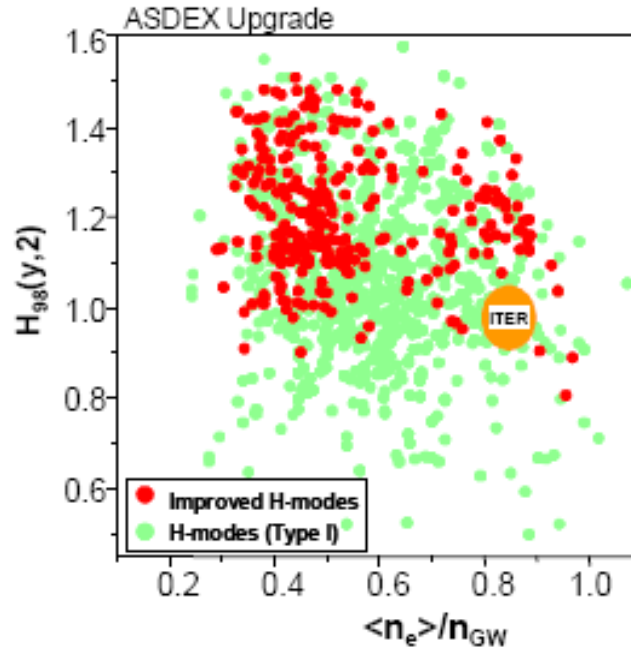
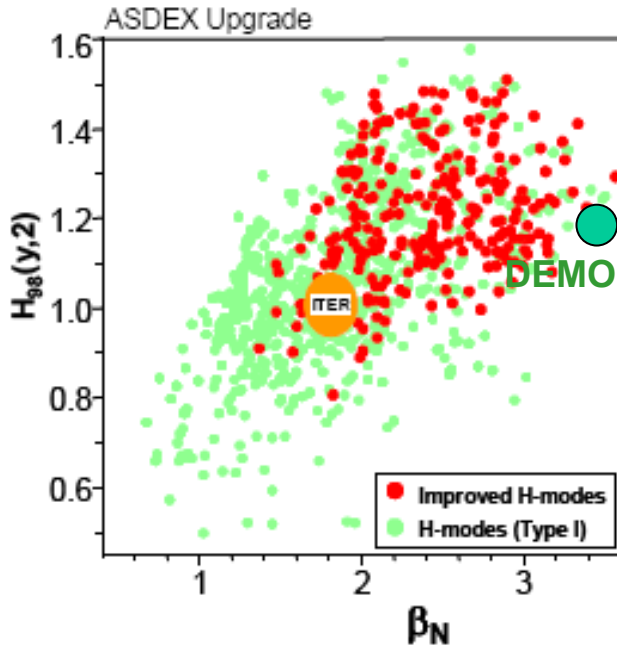


Reversed shear, ITB discharges:

- hollow current profile
- high $\beta_N H/q_{95}^2 > 0.25$ only transient
- control of pressure and current delicate
- high bootstrap fraction $> 60\% \rightarrow ss$
- transport bifurcation



Improved H-mode: Performance and operational range



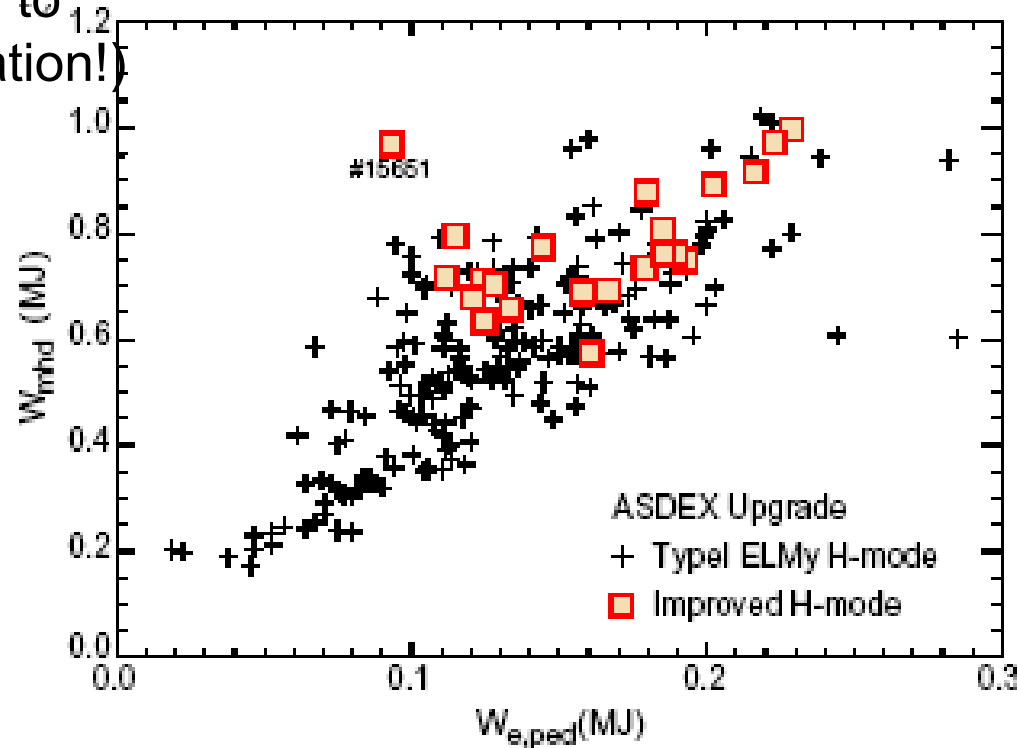
similar correlation of density peaking to v^*

Sips EX/1-1, Weisen EX/8-4

- $\beta_N = 2-3.3$ and $H_{98(y,2)} = 1-1.5$
- β_N above 3 achievable at $q_{95}=3-5$
- operating at ITER collisionality and at densities close to Greenwald
- stationary on several current diffusion times

Improved H-mode: **Confinement**

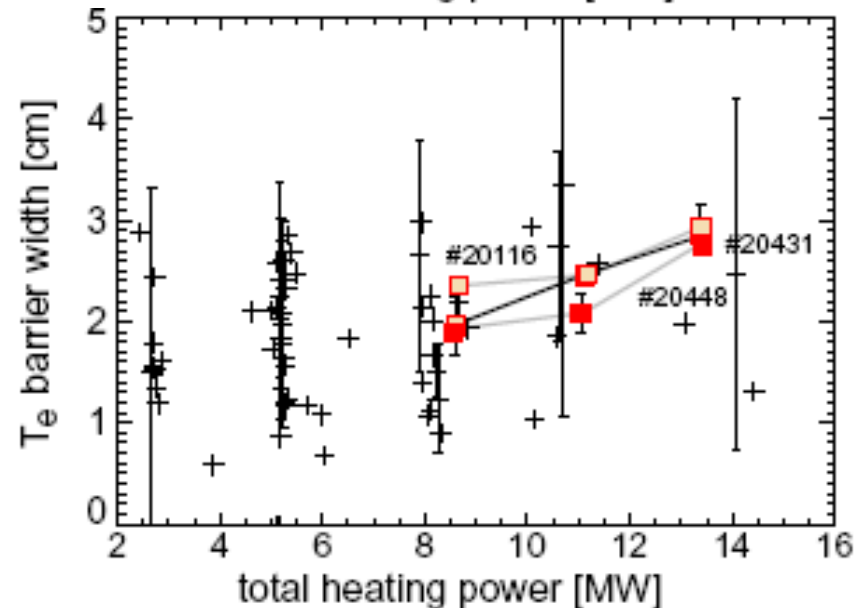
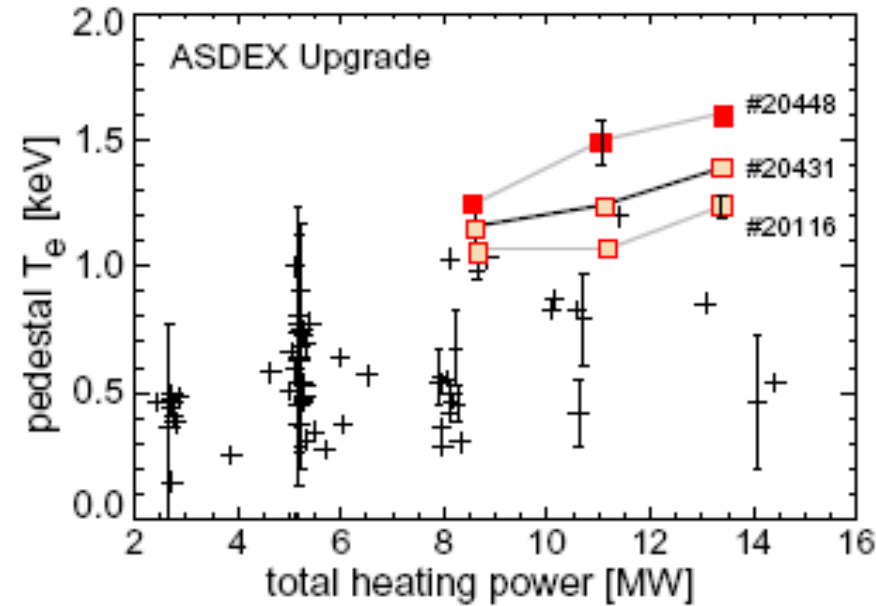
- heat transport given by TEM/ITG turb.
 - stiff temperature profile
 - plasma energy connected with pedestal pressure (pedestal energy)
- confinement improvement weakly correl. with more peaked density profiles
 - flatter density profiles anyway due to central heating (impurity accumulation!)



Suttrop EX/8-5

Improved H-mode: **Confinement**

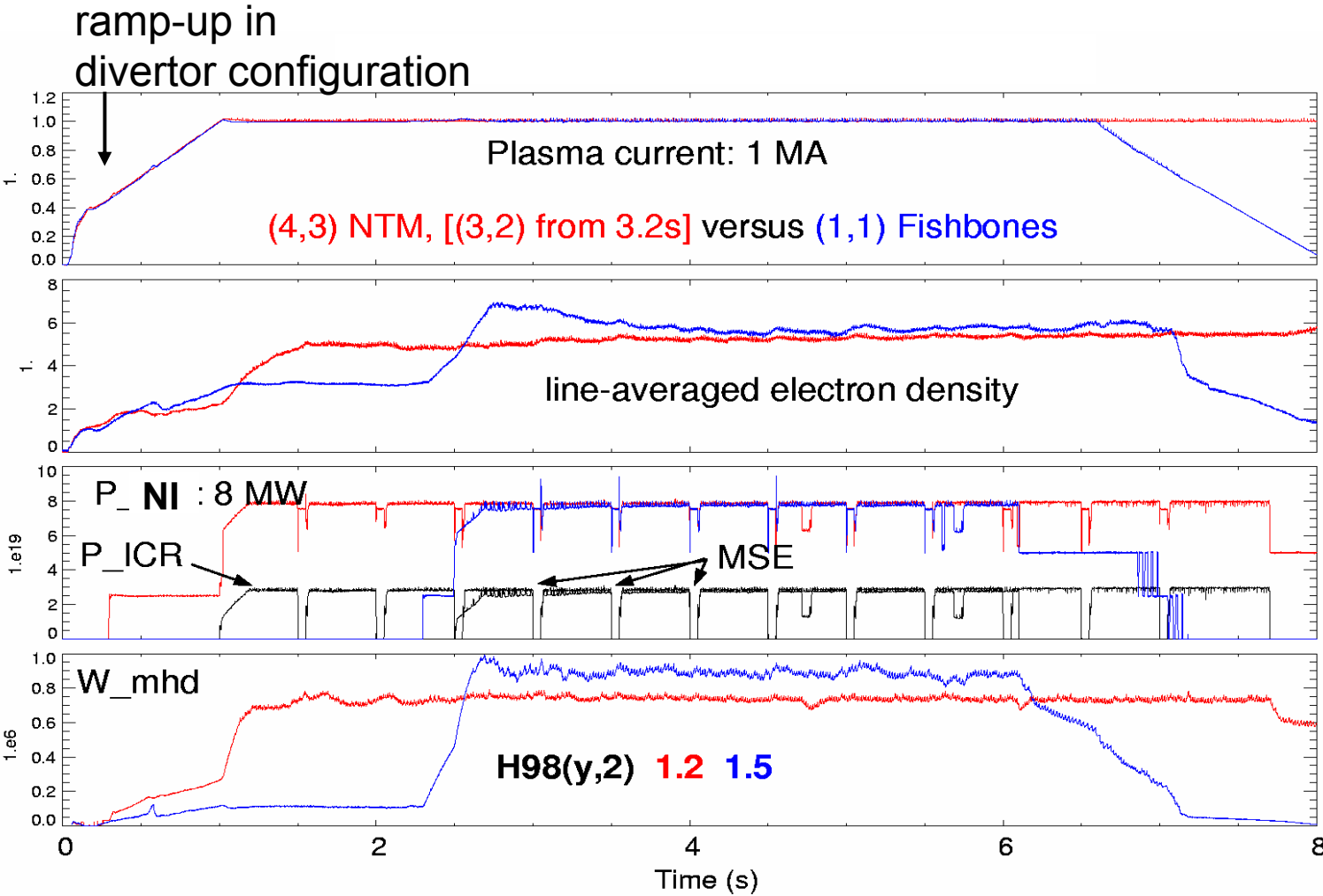
- heat transport given by TEM/ITG turb.
 - stiff temperature profile
 - plasma energy connected with pedestal pressure (pedestal energy)
- confinement improvement weakly correl. with more peaked density profiles
- pedestal top pressure enhanced
 - increases stronger than $P_{add}^{0.3}$
 - predominantly $T_{e,i}$ rise due to broader barrier width
 - $\nabla p_{barrier} \approx const.$



Suttrop EX/8-5, Maggi IT/P1-6,

Improved H-mode: Scenario development

- **early** versus **late** heating: **performance increase**



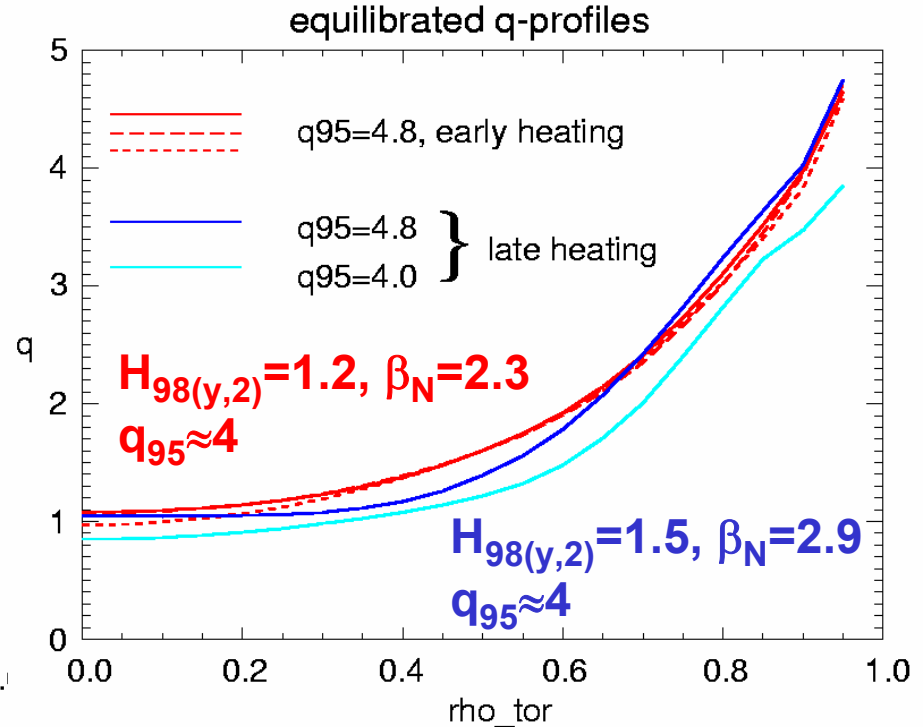
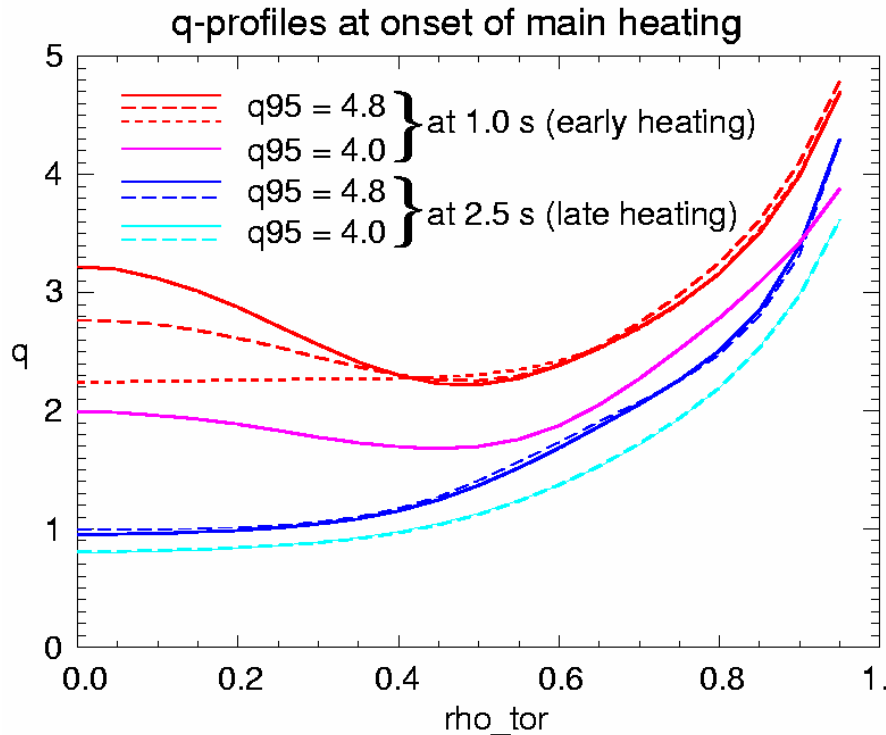
$q_{95} \approx 4.8$
 $T_{e,i}(0) \approx 5.2 \text{ keV}$
 $T_{e,i}(0) \approx 4.5 \text{ keV}$

β_N and $H_{98(y,2)}$ differences disappear at higher heating powers

Stober EX/P1-7

Improved H-mode: Influence of q-profile

- Scenarios with limiter /divertor ramp-up, **early** / **late** heating:
effect on q profile

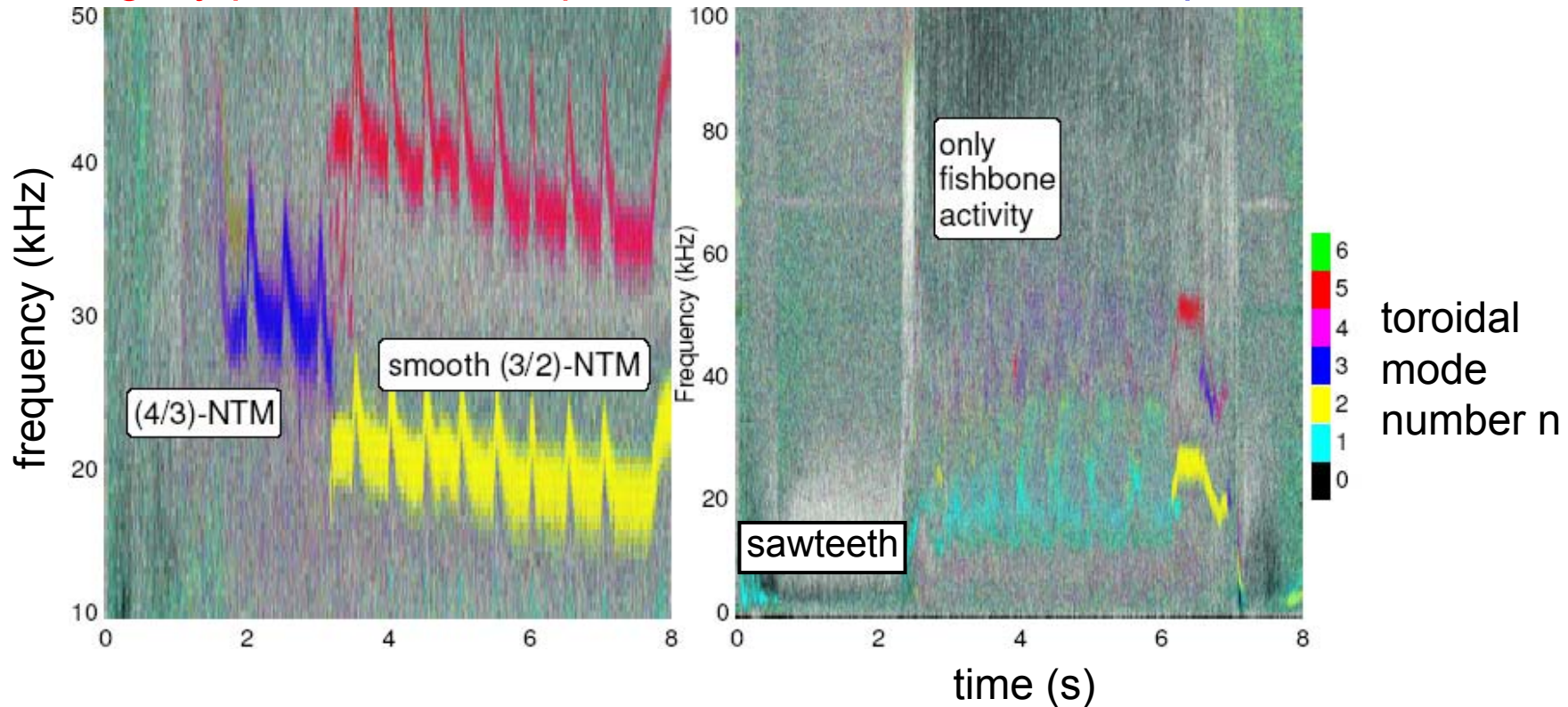


Stober EX/P1-7

Improved H-mode: Influence of q-profile

- different MHD behaviour: both clamp the current profile

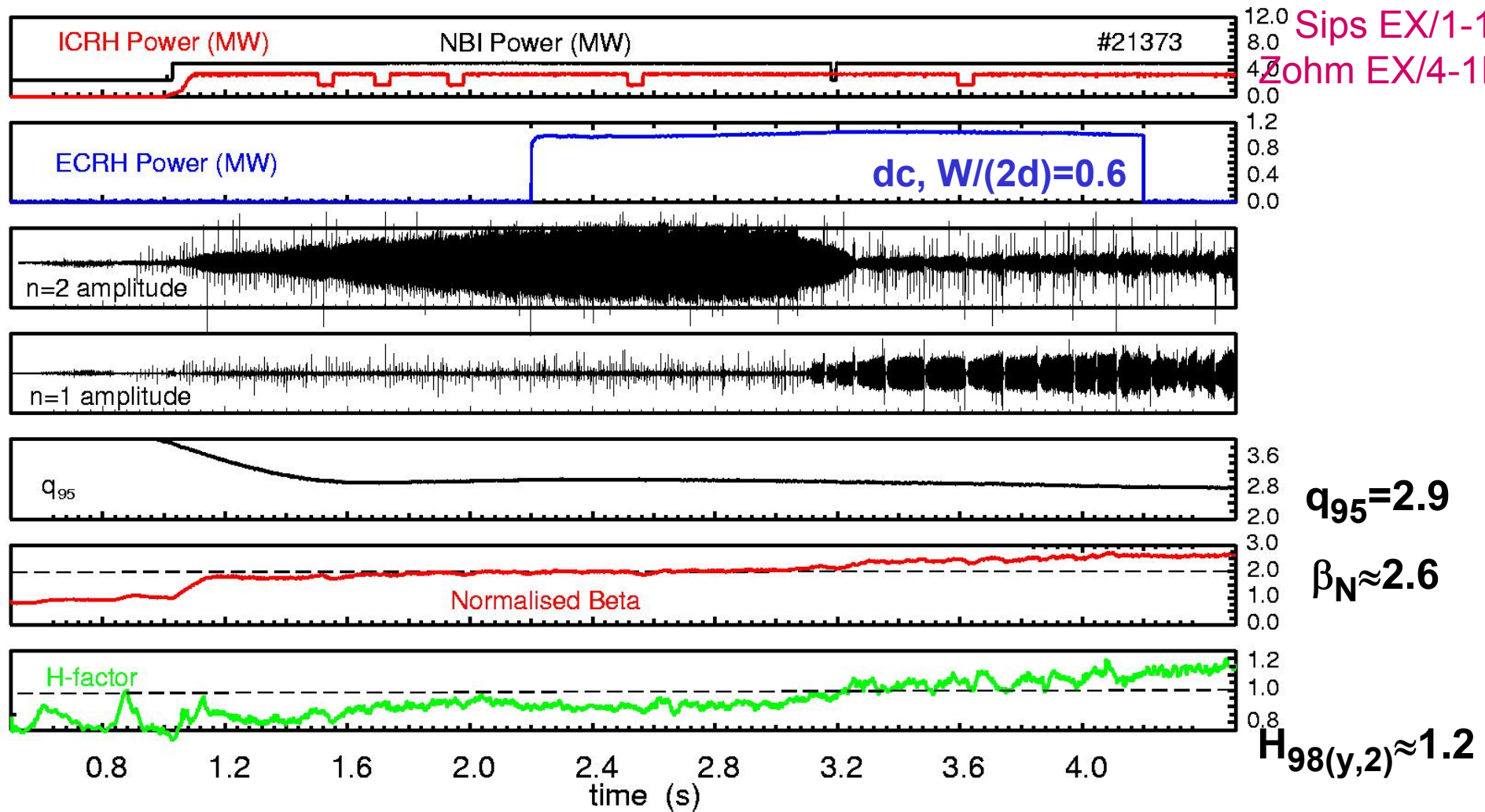
- slightly peaked current profile
- flat central current profile



- influence on transport:
 - theory tells us $R/L_{Ti} \sim s/q$
 - both quantities up to 25% enhanced for flat q-profile
 - in agreement with threshold from GS2 ($\gamma_{max} = \omega_{ExB}$)
- edge pressure increased in case with flatter q-profile

Stober EX/P1-7

Improved H-mode:(3,2) NTM suppression with ECCD



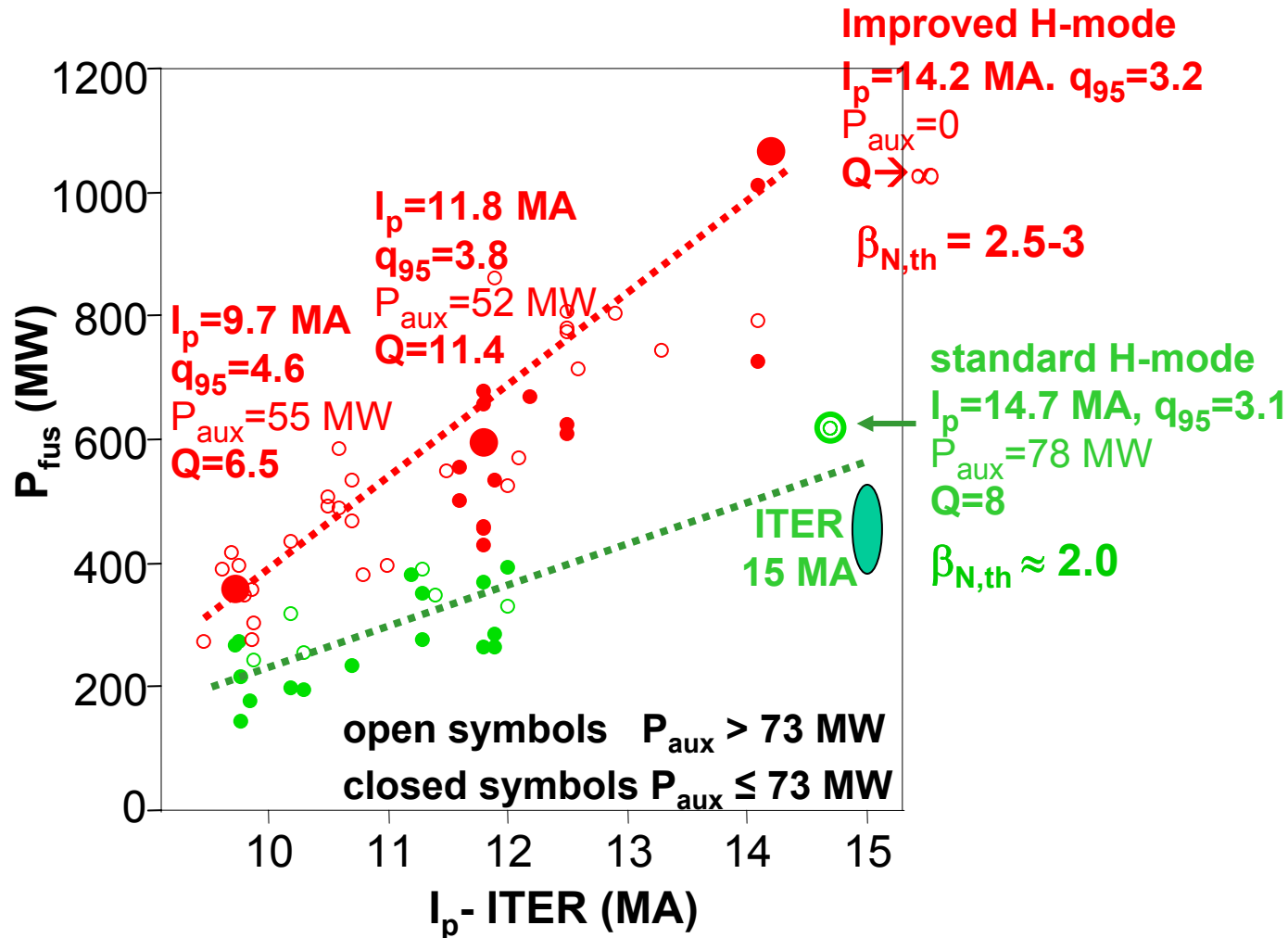
- at low $q_{95} < 3.5$ large (3,2) NTM can develop → strong impact on confinement
- after stabilization transition to fishbone activity → enhanced performance

Improved H-mode: predictions for ITER

Extrapolation to ITER using AUG kinetic profile shapes and IPB98(y,2) scaling

q_{95}, β_N as in AUG, $\langle n \rangle/n_{GW}=0.85, T_e=T_i \rightarrow P_{fus}$

\downarrow
 P_{aux}, Q



Sips EX/1-1

Substantial progress for the benefit of ITER was achieved



- AUG focuses on integrated ITER scenarios and performance beyond reference
- Understanding of anomalous transport and turbulence (TEM, ITG) proceeds:
ITER: peaked density profiles and benign high-Z accumulation to be expected
- Fast ions: - losses caused by MHD and anomalous diffusion important
- off-axis NBCD above a certain turbulence level questionable
- Modulated ECCD needed for NTM stabilization of ITER reference scenario
- ELM (pacemaking) and disruption mitigation (gas injection) schemes evolve
- high-Z walls compatible with tokamak operation modes
 - impurity sputtering source by ICRF accelerated impurities critical
 - accumulation control by ELMs and central heating (α -particles) afforded
- Improved H-mode / Hybrid scenario guides ITER beyond reference scenario
 - ITER parameter range achieved (q_{95} , v^* , n_e/n_{GW})
at $H_{98(y,2)}=1.1-1.5$ and $\beta_N=2.5-3.5$
 - $Q \rightarrow \infty$ and prolonged pulse length at full current ($q_{95}=3$)
 - $Q=10$ and 1 h pulses at reduced current ($q_{95} \geq 4$)
 - heating power of 73 MW may not be sufficient to achieve $\beta_N \approx 3$ for IPB98(y,2)

Future AUG hardware extensions

- Internal coils
 - besides RWM control many other applications ($f=1/10$ kHz); 2007-9
 - compatibility w. RWM control;
 - compatibility with heating systems
 - diagnostic access (YAG,...);
 - relevant diagnostic development
- ICRH antenna fitting to shell
 - installation before shell mounting ?
- LHCD 2008-10
 - 5 MW at 3.7 GHz; 200 kA off-axis CD
 - hardware and manpower from Associations needed
- Stabilising shell 2009-10



- **Main aim is to establish the physics base for ITER (and DEMO):**
 - consolidation of the 'standard' H-mode scenario
 - exploration of ,advanced' modes beyond standard scenario
- **Understanding of physics elements**
 - **transport** Angioni EX/8-5Rb, Conway EX/2-1, Jenko EX/8-5Ra, Weisen EX/8-4
 - **fast particles and NBCD** Günter EX/6-1, McCarthy TH/P3-7
 - **H-mode edge and ELM tailoring** Neuhauser EX/P8-2, Chankin TH/P6-15,
 - **disruption mitigation** Pautasso EX/P8-7 Scott TH/1-1
 - **MHD control with ECCD** Zohm EX/4-1Rb, Yu TH/P3-13, Merkel TH/P3-8
 - **tungsten wall and divertor** Dux EX/3-3Ra, Schmid EX/3-3Rb
- **Integration into improved scenario beyond reference**
 - **Improved H-mode (Hybrid scenario)** Sips EX/1-1, Suttrop EX/8-5, Stober EX/P1-7, Maggi IT/P1-6,